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Iowa Electric Light & Power Co.

RECIP. NAME

RECIPIENT AFFILIATION

DENTON.H.R.

Office of Nuclear Reactor Regulation

SUBJECT: Forwards revised ASME inservice testing program for pumps & valves, June 1978+June 1982, incorporation results of 800124-25 meeting w/NRC, Answers to unresolved questions from

meeting encl. (see reports for Attach. 2 - 8005200496) DISTRIBUTION CODE: A0018 COPIES RECEIVED:LTR \_\_ ENCL: 1\_

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|           | 17 ENGR BR      | 1 1        | 18 REAC SFTY BR   | 1 1       |
|           | 19 PLANT SYS BR | 1 1        | 20 EEB            | 1 1       |
|           | 21 EFLT TRT SYS | 1 1        | EPB-DOR           | 17 1      |
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| EXTERNAL: | 03 EPDR AL SE   | 1 1        | 04 NSIC           | 1. 1      |
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## Iowa Electric Light and Power Company

May 14, 1980 LDR-80-140

LARRY D. ROOT ASSISTANT VICE PRESIDENT NUCLEAR GENERATION

> Mr. Harold Denton, Director Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, D. C. 20555

Dear Mr. Denton:

Attached is our revised Inservice Testing submittal incorporating the results of the January 24 and 25, 1980 meeting between Iowa Electric and the NRC.

Attachment 1 consists of answers to questions developed during the January 24 and 25 meeting and left open for resolution by Iowa Electric. The numbering of the answers corresponds to the numbering of the items in the meeting notes. Relief Request Basis' page references are to the Requests and Justifications for Testing Relief Section of the Valve Testing Program, part of attachment 2 of this submittal.

Attachment 2 consists of the revised ASME Inservice Testing Program for pumps and valves. Revised pages are identified by a revision date (10/78 or 5/80) at the bottom of the page. All changes agreed upon in the January 24 and 25 meeting are included in the revised program. The Testing Program Period of June 1, 1978 to July 1, 1982, includes the normal 40 month program period plus an adjustment of nine months for the down time involving safe end repair.

Very truly yours,

Assistant Vice President

Nuclear Generation

LDR/BAL/mz Attachments

cc: K. Meyer w/o

- D. Arnold w/o
- L. Liu w/o
- S. Tuthill w/o
- B. Lacy w/o
- D. Mineck w/o
- J. Van Sickle w/o
- G. Van Middlesworth w/a
- T. Kevern (NRC) w/a
- K. Harrington w/2 a

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#### IE and NRC Meeting on IST

#### January 24 & 25, 1980

#### Answers to Unresolved Questions

## B. RHR Service Water (Iso M-113)

1. Check valves 'V-13-4 and V-13-15 are not considered to be safety related and have been eliminated from the program.

## C. Nuclear Boiler (Iso M-114)

1. MO-4441 and MO-4442 cannot be partial stroked at power. Relief Request Basis is on Page 7.

#### E. Control Rod Hydraulic System (Iso M-118)

 Basis for not exercising CV-1849 and CV-1850 during cold shutdown is stated in Relief Request Basis on Pages 22 and 23.

# F. Residual Heat Removal (Iso-119)

- MO-1900 and MO-1901 have been reclassified as Category "A"; however, stroking of these valves will be done only at cold shutdown and refueling outages in accordance with Relief Request Basis on page 25a.
- 3. MO-1933, MO-1934, MO-1970, MO-1902 and MO-1905 will remain Category "B". Seat leakage would not prevent them from fulfillment of their function.

## G. Core Spray (Iso M-121)

- 2. MO-2104, MO-2124, MO-2112 and MO-2132 will remain Category "B". Seat leakage in the closed position would not create a safety hazard.
- 3. Qy-2118 and CV-2133 will remain category "C". MO-2137 and MO-2117 will provide adequate leakage protection.

# I. HPCI (water side) (Iso M-123)

3. Valve V-23-1 will be disassembled and inspected at each refueling outage. See Relief Request Basis on Page 33a.

# K. RCIC (water side) (Iso M-125)

3. Valve V-25-1. Same as V-23-1 in the HPCI system. See Relief Request Basis on Page 35a.

- 0. MSIV Leakage Control (Iso M-184)
  - 1. MO-8401 A, B, C and D valves are safety related.